

DIII-D's Role in Advancing U.S. Interests in Fusion Energy Development

by
M.R. Wade

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MFE Priorities
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International Team



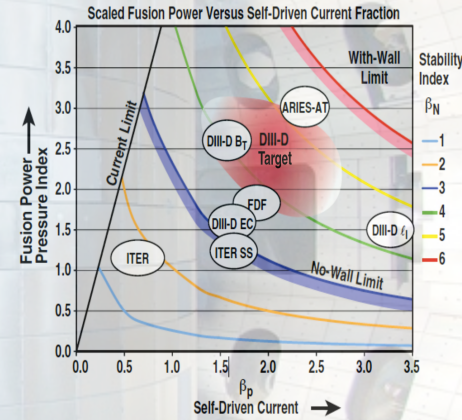
Physics Measurements

- X-pi SXR Camera
- The current array
- FIDA
- Bolometers
- IR cameras
- Fast ion collectors
- SXR
- Filtriscopes
- MSE
- FIR & g w scattering
- BES
- SPRED
- Vertical scanning probe
- Magnetics
- ECU
- MID spectrometer
- Visible bremsstrahlung
- Gamma detectors
- Fast wave reflectometer

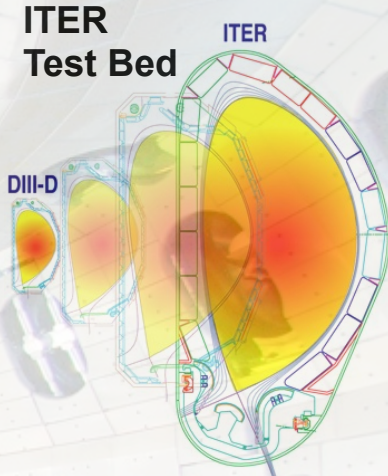
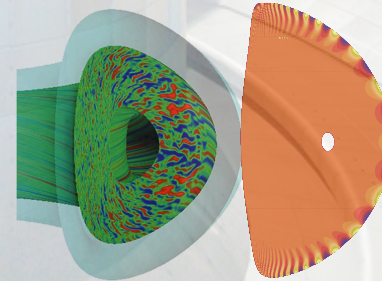
DIII-D Diagnostics

- Thomson scattering
- Coherence Imaging
- Lithium beam spectroscopy
- CF swing probe
- DISRAD
- TAFIF
- CECE
- CER
- VUV cameras
- ASDEX gauges
- Visible cameras
- Fast framing camera
- DIBS
- DIMES
- ECE
- NPAs
- Langmuir probes
- Reflectometers
- Fast Ion Loss Detector
- Interferometers

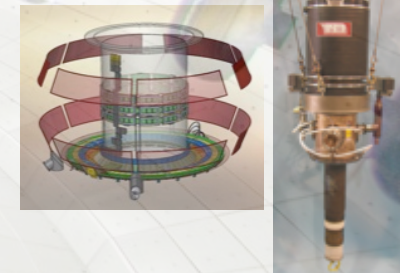
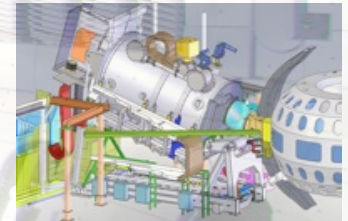
Scenario Development



Theory & Simulation



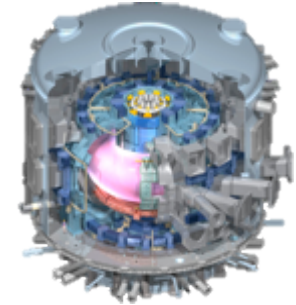
Plasma Control



DIII-D Is a Critical Element in Advancing and Informing U.S. Program Elements on the Path to Fusion Energy

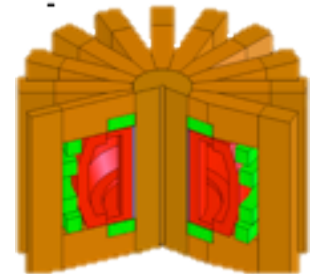
DIII-D

Resolve key physics issues for ITER design and performance optimization



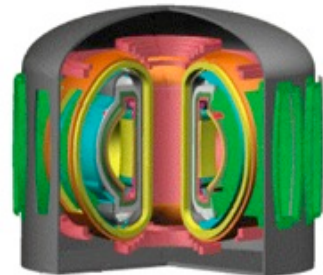
ITER

Inform decisions on next-step facility aimed at fusion energy development



FNSF

Provide physics basis for steady-state power plant



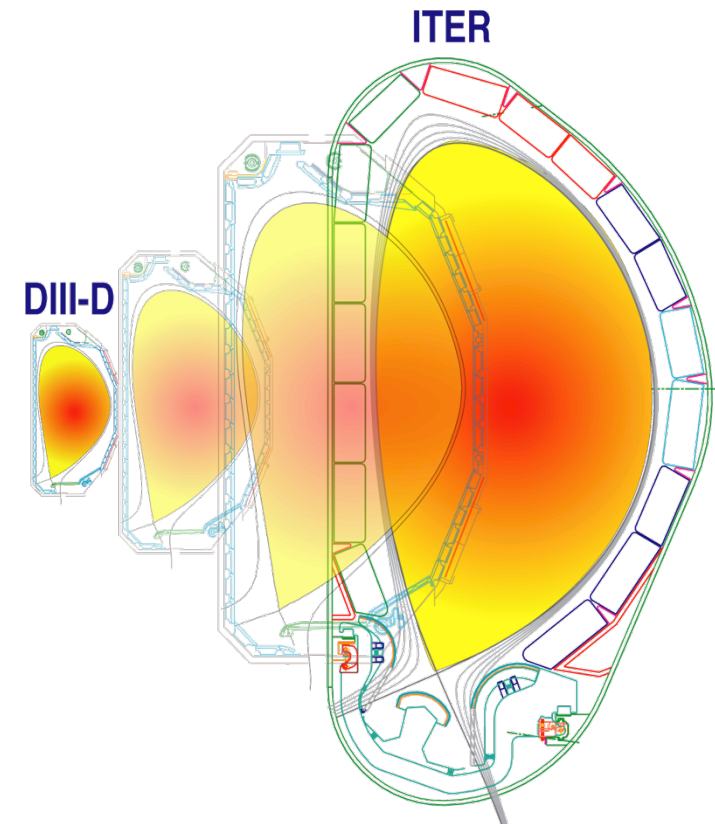
DEMO

Train the next generation of scientists for ITER and next-step devices



DIII-D is Uniquely Positioned to Resolve Key Physics Challenges and Ensure ITER's Success

- Qualify techniques for ELM control in ITER
- Develop reliable means to avoid and mitigate damaging effects of ITER disruptions
- Determine limitations imposed by new superconducting coil constraints
- Develop reliable plasma control techniques consistent with ITER capabilities
- Develop and quantify performance of plasma regime expected in ITER (dominant electron heating, low NBI torque)
- Provide physics basis for improved confidence in ITER heat flux control plans

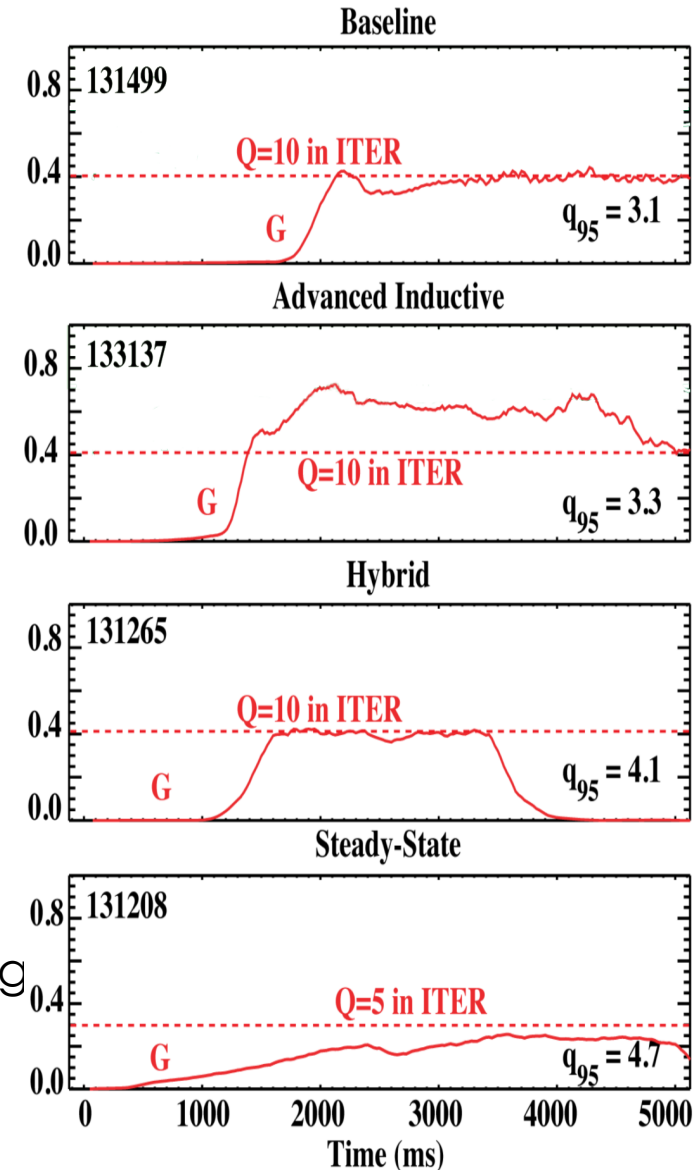


**DIII-D: U.S.'s ITER Simulator
~1/4 size ITER Prototype**

Facility capabilities and responsive team enable rapid answers to ITER urgent requests

DIII-D Capabilities Indicate Opportunity for Extending the Technical Reach of ITER

- **ITER baseline scenario provides minimal level of performance for U.S. science needs**
 - DIII-D has demonstrated performance well beyond this level
- **Potential exists on ITER for:**
 - $Q = 10$ at lower plasma current (less risk to ITER)
 - $Q > 10$ inductive (ignition possible)
 - $Q > 5$ steady-state (FNSF/DEMO proof-of-principle)
- **Such capabilities on ITER would expedite development of fusion energy**
 - Improved understanding of key processes in burning plasmas (e.g., transport, energetic particles, 3D fields)
 - Define path to performance optimization in DEMO
 - Control tool development and strategy testing

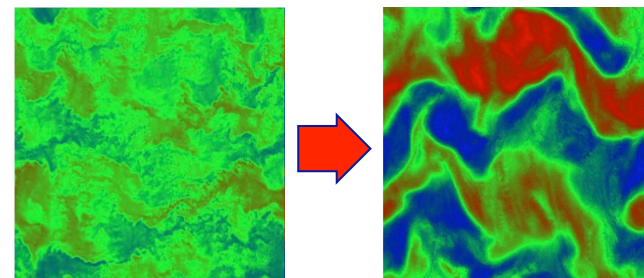


Fusion Gain Figure of Merit
$$G = \beta_N H_{89} / q_{95}^2$$

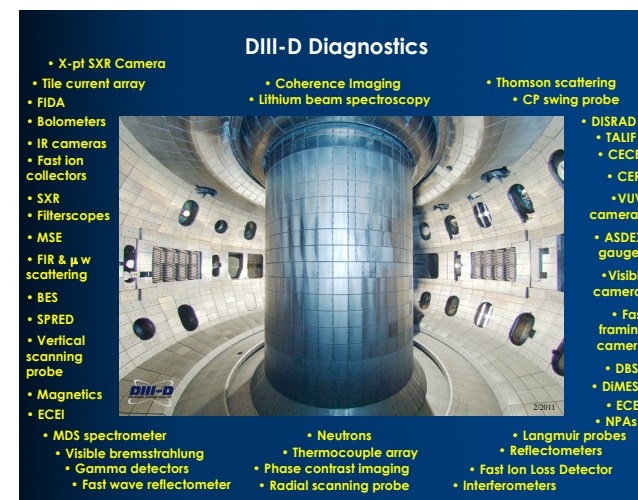
DIII-D Will Enable U.S. to Take Leadership Positions in ITER Science

- **ITER is potentially impacted by new physics**
 - Small gyroradius (ρ^*) effects
 - Transport in burning plasma regime
 - Fast ion instabilities in “sea” of modes
- **DIII-D provides excellent scientific platform for developing physics models of these effects**
 - Flexible control tools and geometry
 - Comprehensive diagnostic set
 - Access to ITER regimes
- **DIII-D is an outstanding facility for training the next generation of fusion scientists and can help build the U.S. team for exploitation of ITER**

Scale of turbulent eddies predicted to change in burning plasma regime

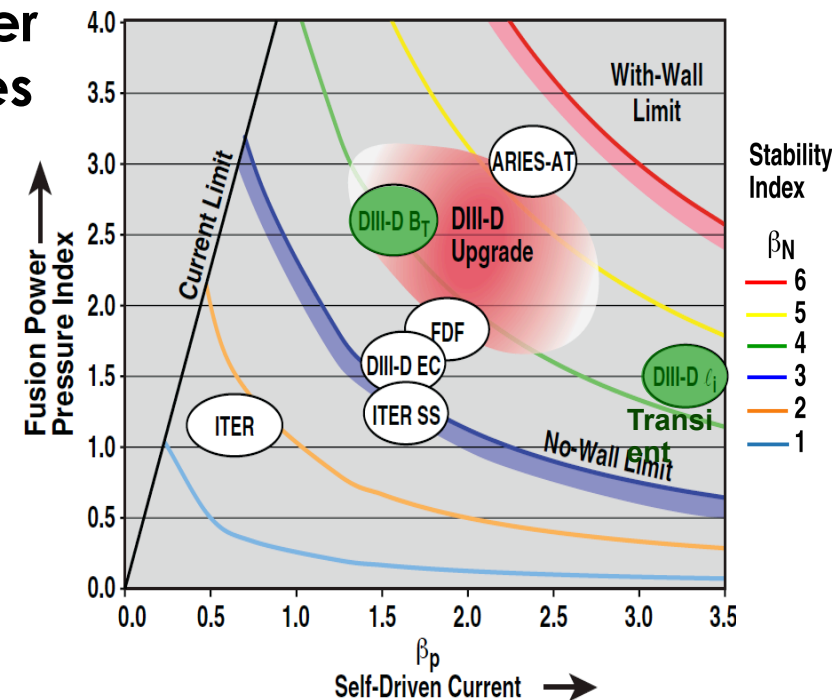
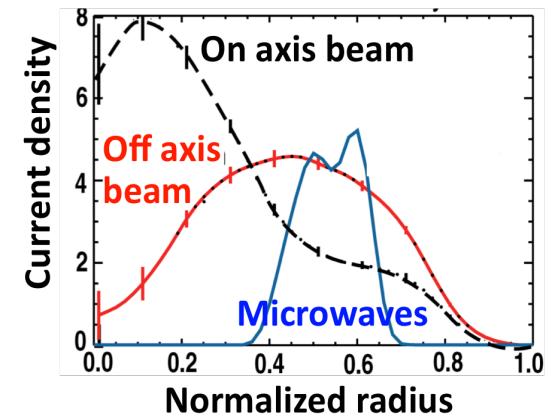


Comprehensive diagnostics



DIII-D Capabilities Enable US to Maintain Leadership on Path to Fusion Energy Beyond ITER

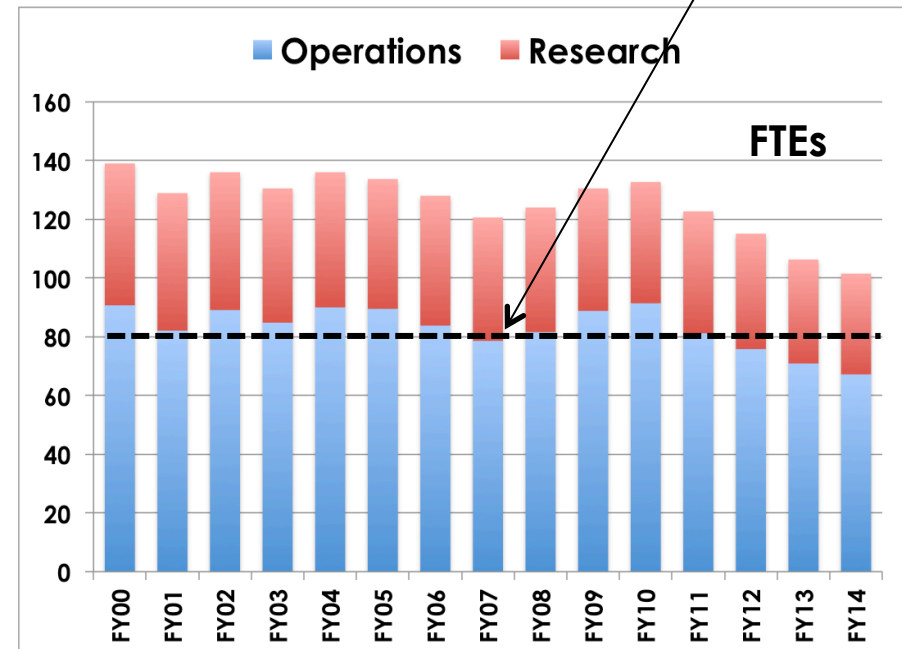
- **Steady-state operation raises new challenges:**
 - Broad profiles to enable high beta access and full non-inductive operation
 - Detached boundary to eliminate erosion
- **DIII-D's flexible heating and current drive offer basis to inform decisions on next step devices**
 - Off-axis current drive provides access to regimes of interest for FNSF and DEMO
 - Sufficient power to probe stability limits
 - Dominant electron heating to assess impact of transport
- **Shape flexibility allows exploration of using boundary geometry to optimize divertor detachment**



Additional Funding is Required to Properly Exploit DIII-D Capabilities in Pushing Fusion Energy Forward

Critical Mass

- Available budget is near critical level to support safe, reliable operations
- FES guidance of 10 weeks operations in FY13 will limit program's ability to advance fusion energy development
 - Requires hard choices on research priorities
- Long-term impact will be loss of U.S. leadership in key areas
 - E.g., scenarios, transport, energetic particles, 3D fields



“Existing technical staff appears marginally adequate to support the combination of operations, maintenance of aging equipment.” - DoE DIII-D Facility Review Panel, May 2008

With the guidance budget, DIII-D is the only operating U.S facility in FY13-14

Further Upgrades to DIII-D Will Advance U.S. Leadership in Key Areas for Fusion Energy Development

Burning Plasma Regimes

- Dominantly electron heated
- Low torque
- Low fueling & collisionality
- Fast ion physics

Manage stability & transients

- ECCD stability control
- Anticipate, avoid & mitigate disruptions
- 3D optimization: ELMs error fields, RWM

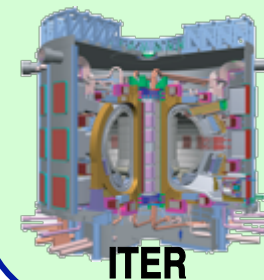
Proposed DIII-D Upgrades:

- EC: 12 MW
- NBI: 30 MW total, 10 MW off-axis
- Advanced 3-D Capability
- New diagnostics for physics validation

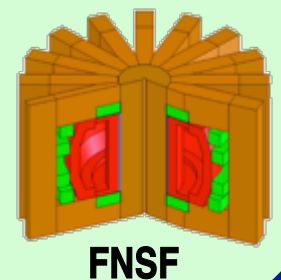
Steady-state Plasma Operation

- Fully noninductive
- Self-driven well aligned currents
- High confinement
- Spread heat & cool exhaust

Compelling Physics Basis for Fusion Energy Development



ITER



FNSF

DIII-D Is a Critical Element in U.S. Preparations for ITER and in Defining Next Facility on Path to Fusion Energy

